ABSTRACT

In the framework of European fusion activities, a high flux neutron source is considered an essential device for testing candidate materials under irradiation conditions typical of future fusion power reactors. To this purpose, IFMIF (International Fusion Materials Irradiation Facility) project represents an important option to provide the fusion community with a source capable of irradiating materials samples at a rate of up to 20 dpa/fpy in a volume of 0.5 l. This is achieved by bombardment of a high-speed liquid lithium target with a 10 MW double deuteron beam which yields a 14 MeV peaked neutron spectrum. Within the engineering design work of the IFMIF/ENEA project, which was concluded in half 2013, ENEA was in charge of the design of the lithium target system based on the so called bayonet backplate (BP) concept, which foresees the possibility to periodically replace only the most irradiated and thus critical component (i.e., the backplate) while continuing to operate the rest of the target for a longer period. With the objective of estimating the lifetime of the BP, a pseudo-transient calculation simulating one year of full power operation has been performed by imposing a non-uniform neutron-induced volumetric swelling strain which evolves in time as a linear function of the accumulated displacement damage (dpa) according to available literature experimental correlations. Dpa damage distribution and time rate have been calculated by ENEA in the framework of an extensive neutron analysis of the target system carried out through the MCNP transport code. The stress field evolution resulting from the increasing swelling deformation has been obtained through an uncoupled thermomechanical analysis carried out by the University of Palermo using a qualified finite-element code which takes into account all thermal and mechanical loads applied to the system during normal operation. In this paper, the results of the above analysis are presented and an estimation of the BP lifetime due to swelling effect is given on the basis of ITER design rules criteria. It is found that, although the bulk structure of the BP is expected to survive several months of continuous irradiation, a further improvement of the system is suggested in order to achieve an optimized configuration.

NEUTRONICS:

- Neutronics:
  - MCNP5 + McDeLicious-11 neutron source (KIT)
  - McCad (KIT) + MCAM (FDS) for CAD/MCNP files conversion

THERMOMECHANICS:

- 3D uncoupled FE model (ABAQUS® code)
- Primary loads: mechanical loads
- Secondary (Q) loads: thermal loads + swelling

Operating conditions:
- Beams power = 10 MW
- Li inlet temperature = 250 °C
- Li inlet velocity = 15 m/s

Values at beginning of nominal operation

<table>
<thead>
<tr>
<th>Path AB - Level A criteria</th>
<th>Values at beginning of nominal operation</th>
</tr>
</thead>
<tbody>
<tr>
<td>Tmax [°C]</td>
<td>283</td>
</tr>
<tr>
<td>P0 / Ps</td>
<td>0.0035</td>
</tr>
<tr>
<td>(P0 + Ps) / (Kme S0)</td>
<td>0.0026</td>
</tr>
<tr>
<td>(P0 + Ps) / S0</td>
<td>0.7835</td>
</tr>
<tr>
<td>EIRC (end of life)</td>
<td>~ 0.5 %</td>
</tr>
</tbody>
</table>

Stress evolution

ITER SDC-IC design rules

\[ (P + Q)_m : \text{primary + secondary membrane stress intensity} \]

\[ S_0 : \text{allowable stress intensity} \]

Based on literature [G. Aiello et al., J. Nucl. Mat., 414 (2011), 52-68]

1) Increase of the Li channel thickness

2) Reduction of the backplate thickness